

SURVEY OF WEAPONS DEVELOPMENT AND TECHNOLOGY

WR708

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SESSION II

- REVIEW OF WEAPONS PHYSICS**
- THEORY OF NUCLEAR EXPLOSIONS**

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Weapons Physics and Nuclear Material

- Several basic nuclear physics concepts and the properties of the nuclear fissile material are very important to the understanding of weaponization
 - The physics of fission
 - Nuclear properties
 - Availability of material
 - How the fissile material is obtained
 - Energy available and energy trades

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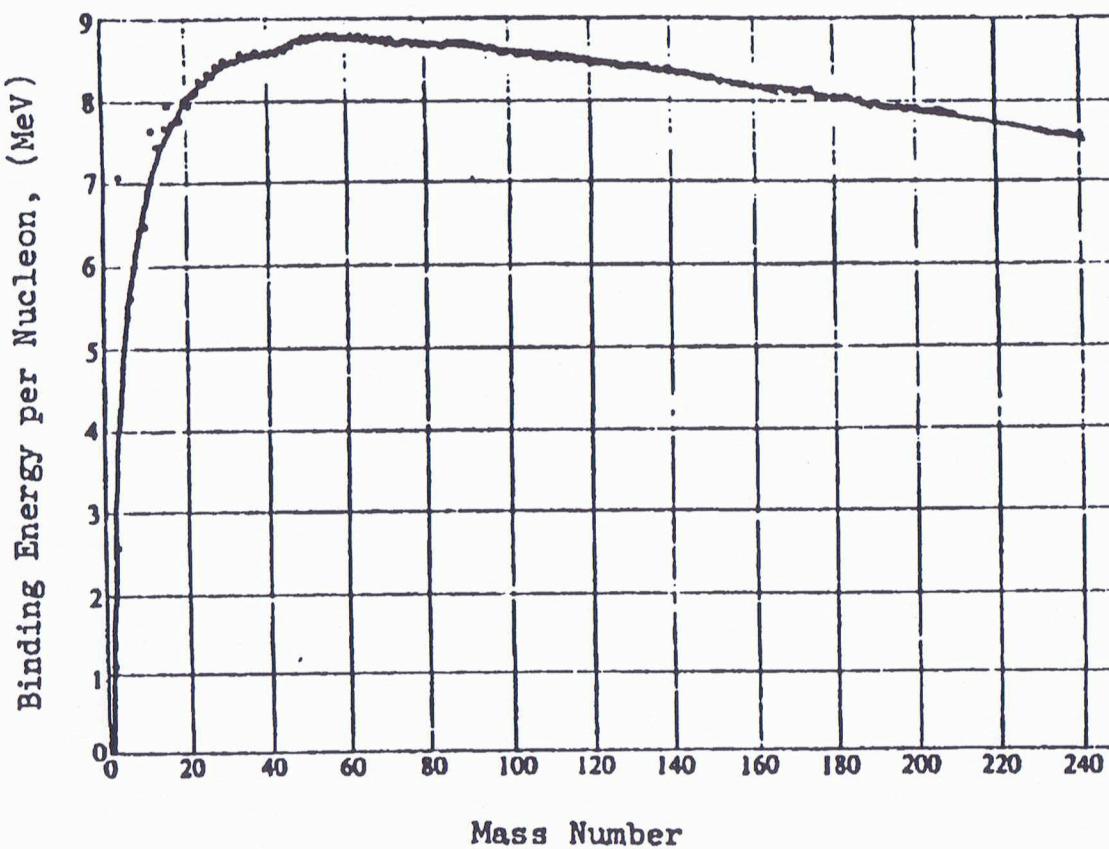
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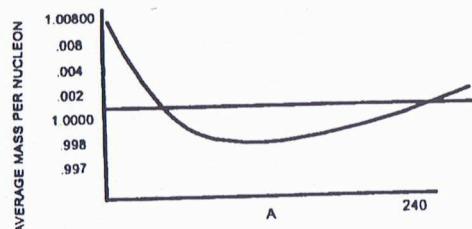
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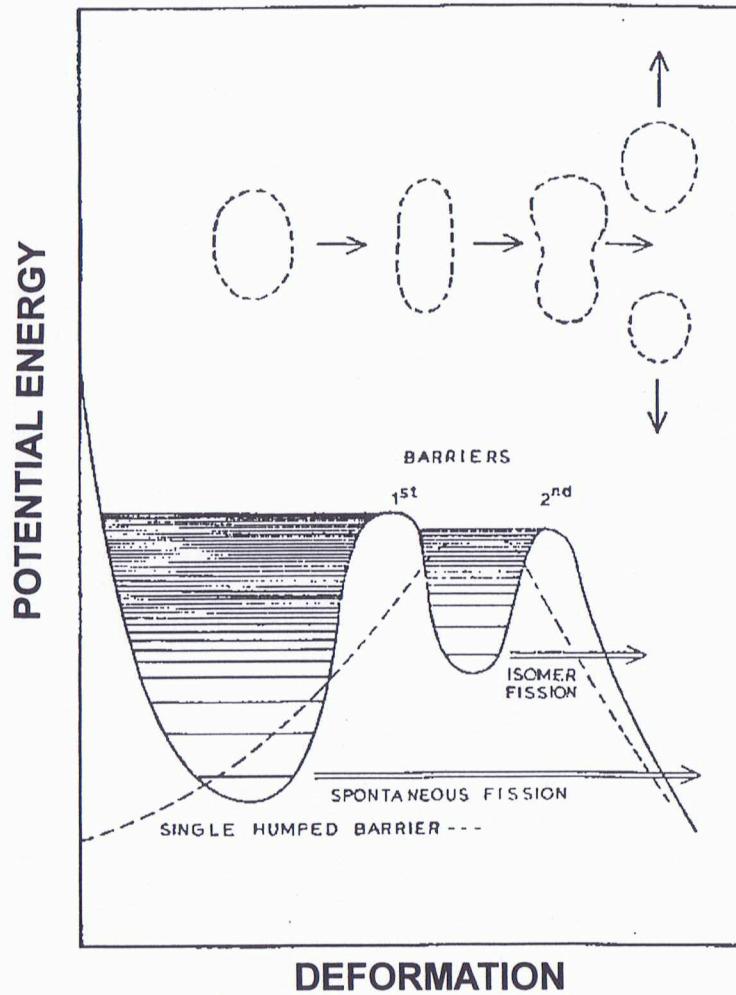
NUCLEAR BINDING ENERGY



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LIQUID DROP MODEL APPLIED TO POTENTIAL BARRIERS



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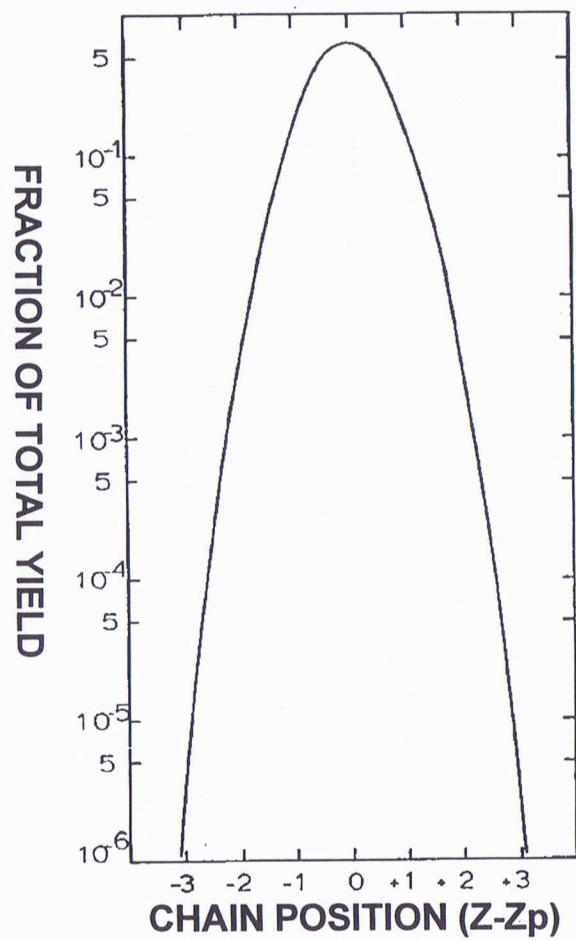
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CHARGE DISTRIBUTION CURVE

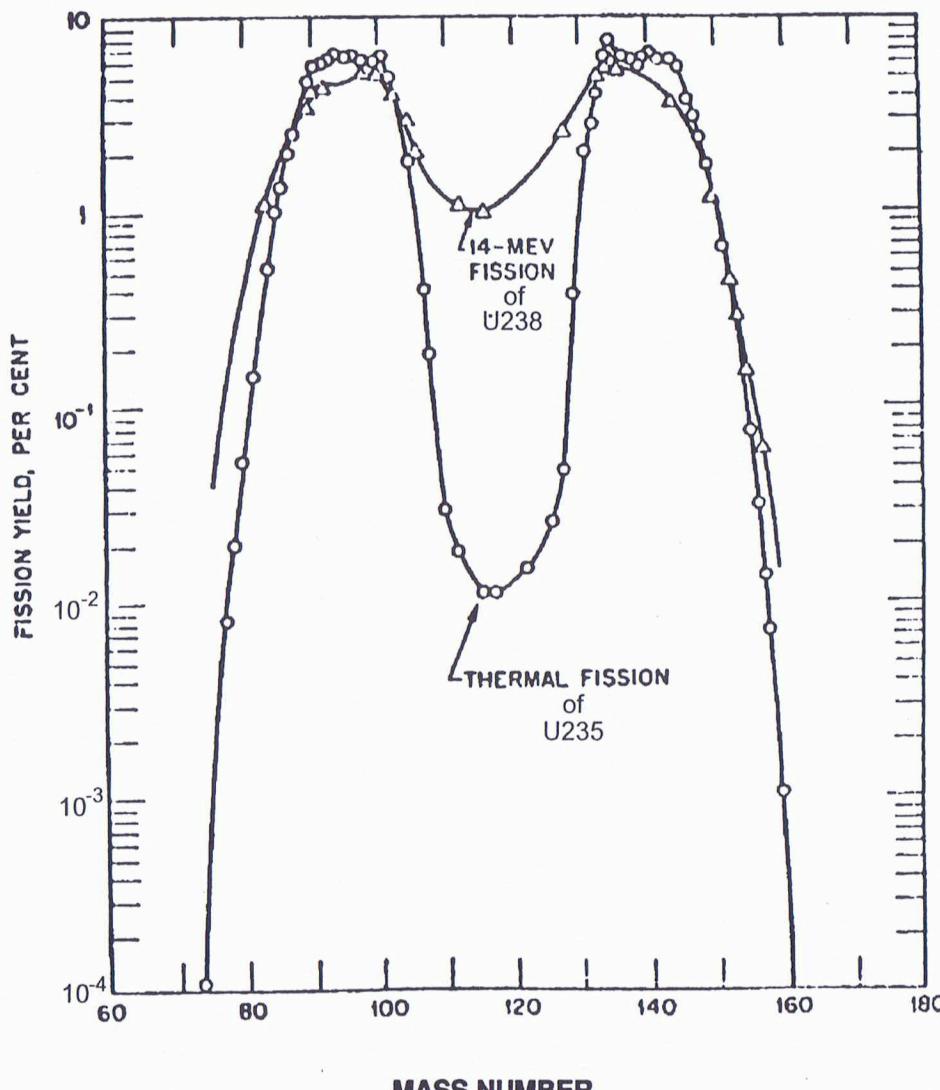


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LIKELIHOOD FOR FISSION FRAGMENT

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Terminology

Asymmetric fission

- division of excited nucleus into two unequal fragments with masses about 100 & 140 amu.

Binary

- division at scission point into two parts.

Cross-Section

- probability that a certain reaction between a nucleus and an incident particle or photon will occur, as in a neutron and U²³⁵ (measured in "barns")

Fission Fragment

- fragment after scission but before prompt neutron emission

Fission Product

- fragment after prompt neutron emission

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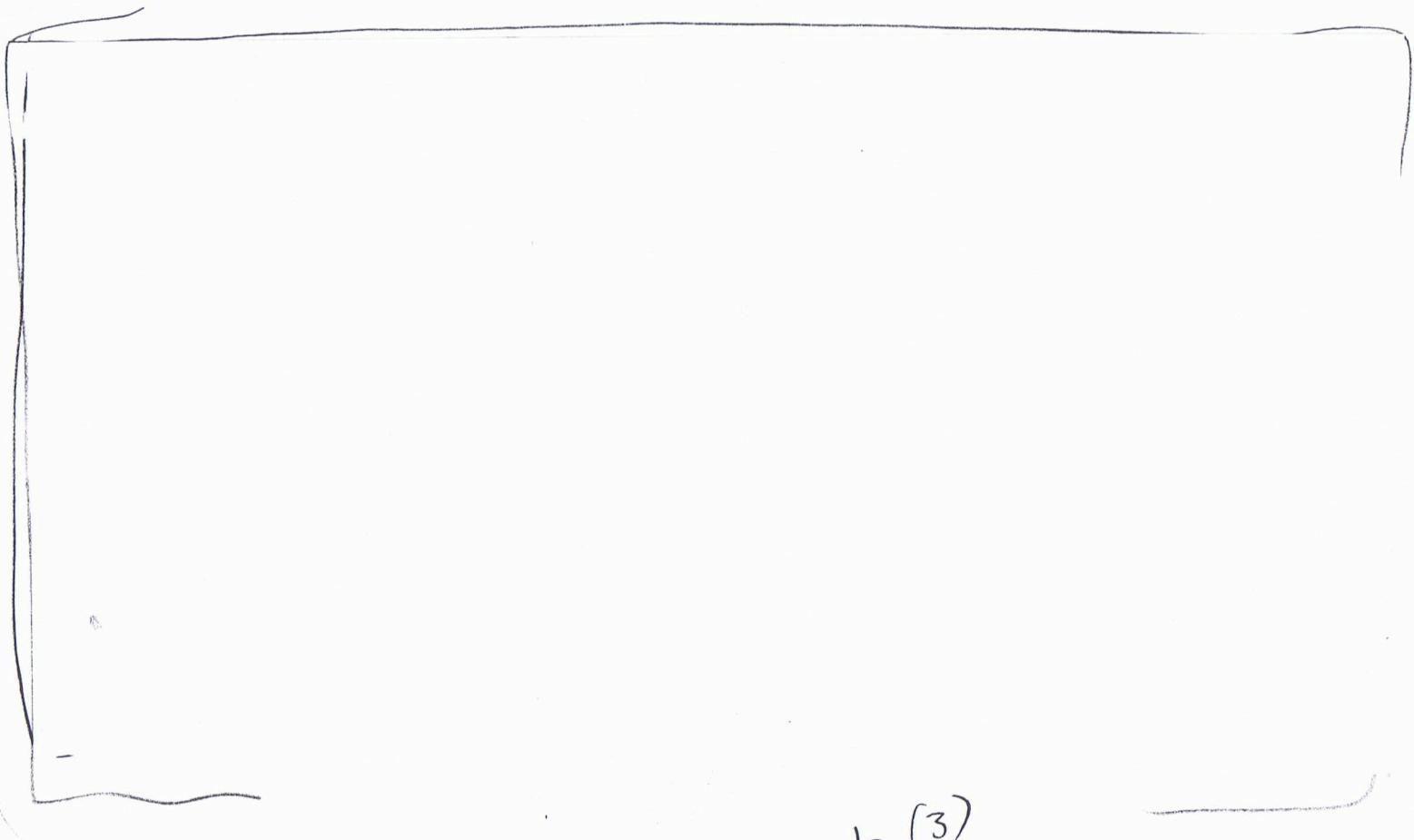
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The Gang of Four

$^{238}\text{U}_{92}$	^{239}Pu
% in nature - 99.27 When the $^{238}\text{U}_{92}$ is extracted, it is called depleted ^{238}U or TUBALLOY or D38 (from UK WWII effort - TUBE ALLOY) Will fission but not fissile Physically separated	% in nature - essentially zero (mine in South Africa) Made in reactor: $n + ^{238}\text{U} = ^{239}\text{Pu}$
^{235}U % in nature - 00.73 Concentrated to 93.5% Called ORALLOY for Oak Ridge Alloy	^{240}Pu % in nature - essentially zero Made by reactor If you leave the ^{239}Pu in "too long," it will absorb a n $\rightarrow ^{240}\text{Pu}$ Spontaneously fissions (originally a problem for pre-ignition)

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CALCULATION OF ENERGY RELEASE



235.0439
1.0087

94.905837
138.906400
.003850
2.017340

236.0526 amu

235.8334 amu atomic mass unit

MASS DEFECT OF .219 amu

$$n = 1.00867 \text{ amu}$$

$$p = 1.00728 \text{ amu}$$

$$e = .00055 \text{ amu}$$

$$(.219 \text{ amu}) (931.4 \frac{\text{MeV}}{\text{amu}}) \approx 204 \text{ MeV}$$

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THE EXAMPLE STARTED WITH



FISSION CHAIN

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THEORETICAL FISSION ENERGY

- THERE ARE $\frac{6.025 \times 10^{23}}{235.0439}$ ATOMS PER GRAM OF $_{92}^{235}\text{U}$
- THEREFORE, 1 kg OF $_{92}^{235}\text{U}$ HAS 2.5634×10^{24} ATOMS
- HENCE, @ 180 MeV PER FISSION 1 kg OF $_{92}^{235}\text{U}$ WOULD PRODUCE 4.6141×10^{26} MeV IF EACH ATOM WERE FISSIONED.
- CONVERTING TO KILOTONS
- $(4.6141 \times 10^{26} \text{ MeV}) (3.824 \times 10^{-26} \frac{\text{kT}}{\text{MeV}}) \approx 18 \text{ kT}$

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FACTORS AFFECTING CRITICAL MASS

- GEOMETRY
- AMOUNT OF MATERIAL
- TYPE OF MATERIAL
- PURITY OF MATERIAL
- SURROUNDING MATERIAL
- DENSITY

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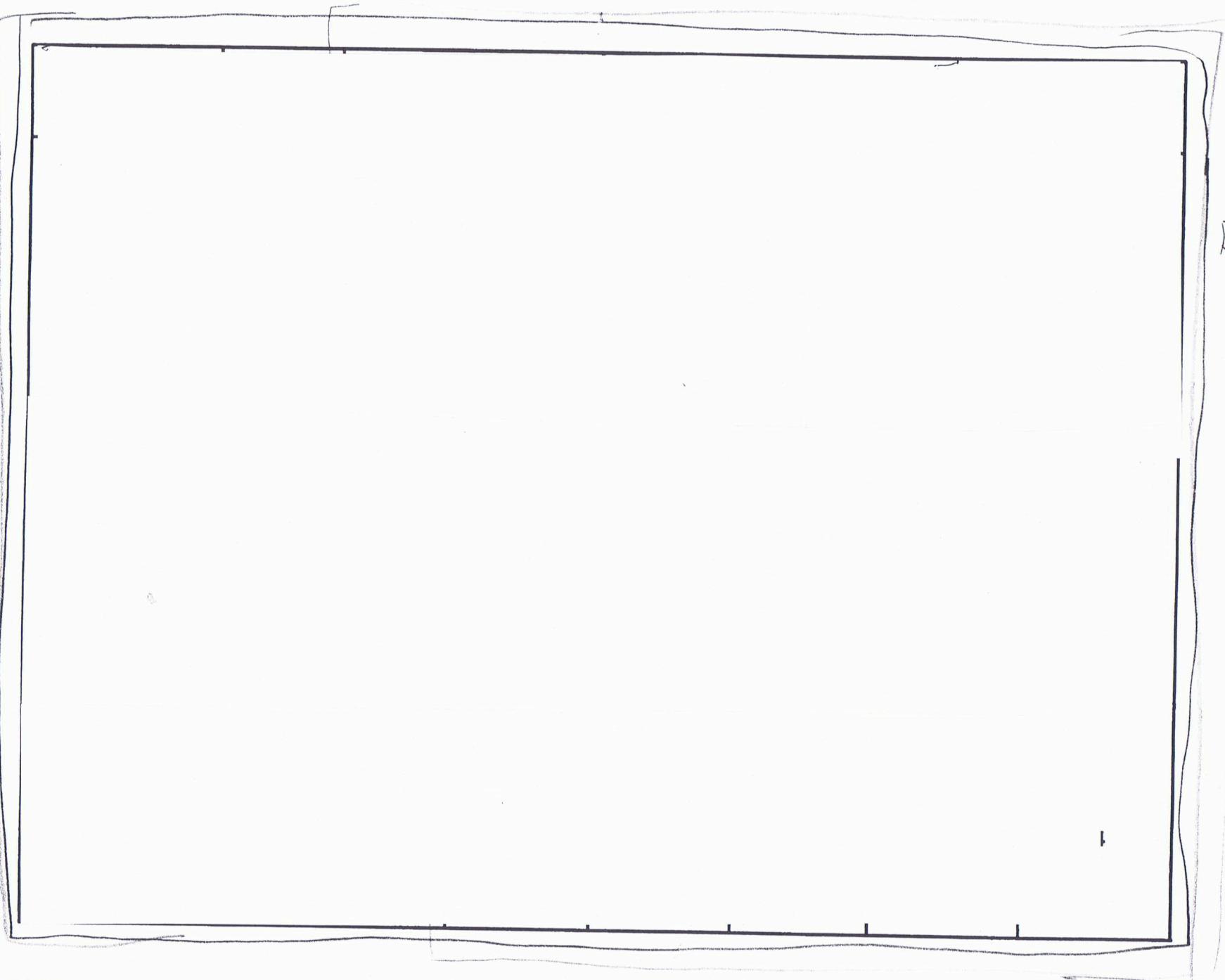
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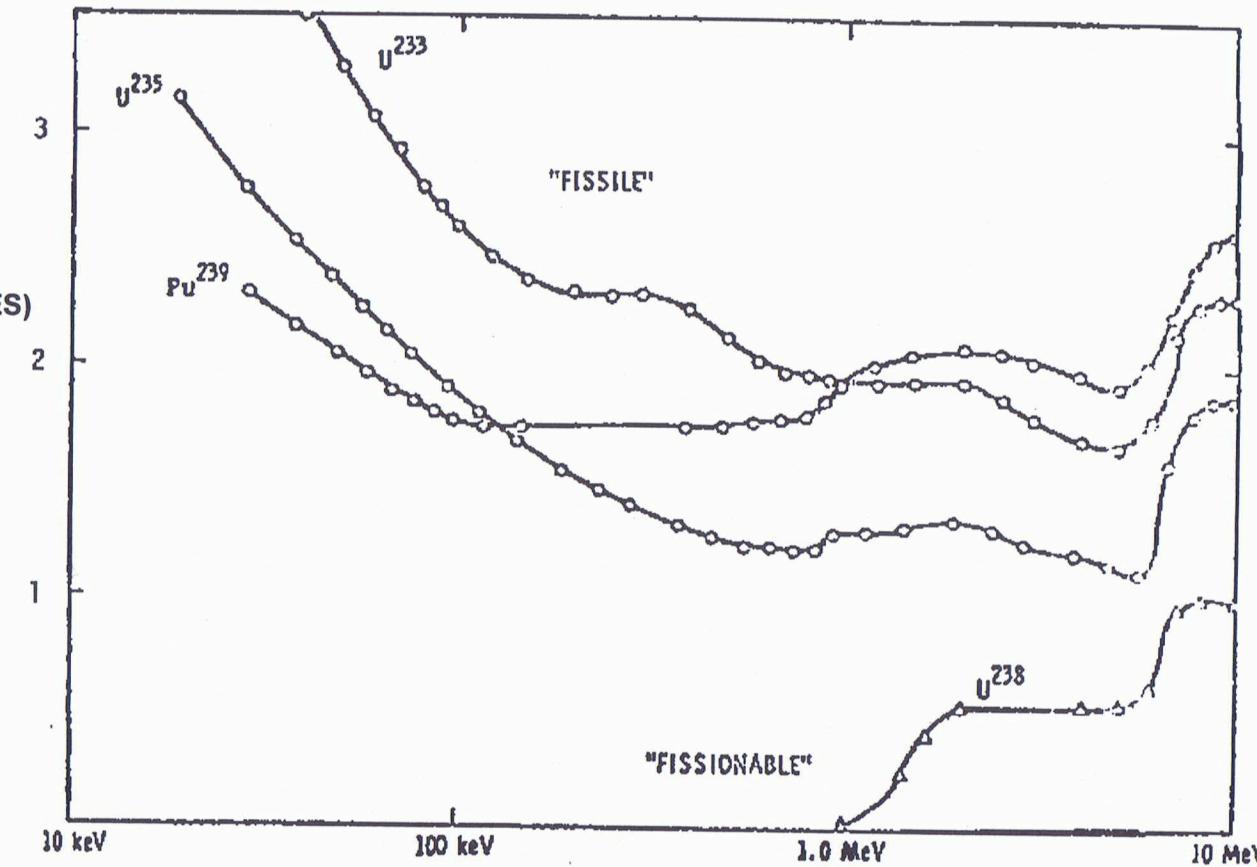
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FISSION CROSS SECTIONS

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INCIDENT NEUTRON ENERGY

NOTE: The thermal neutron energy is not on the chart

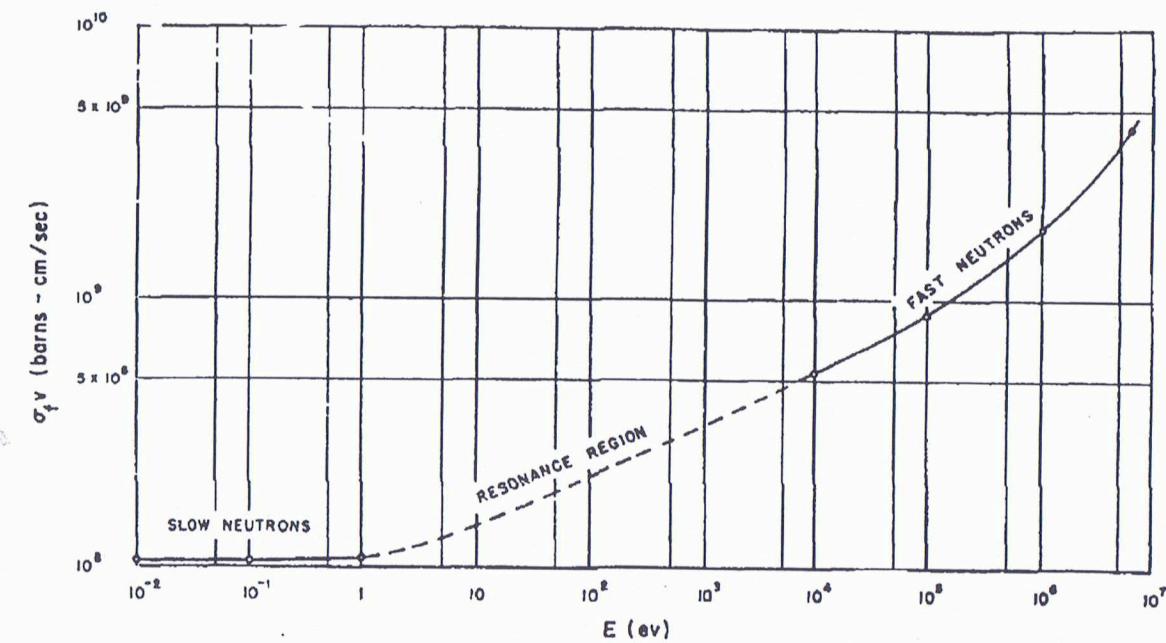
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Variation of Cross Section x Ave. # Neutrons for ^{235}U



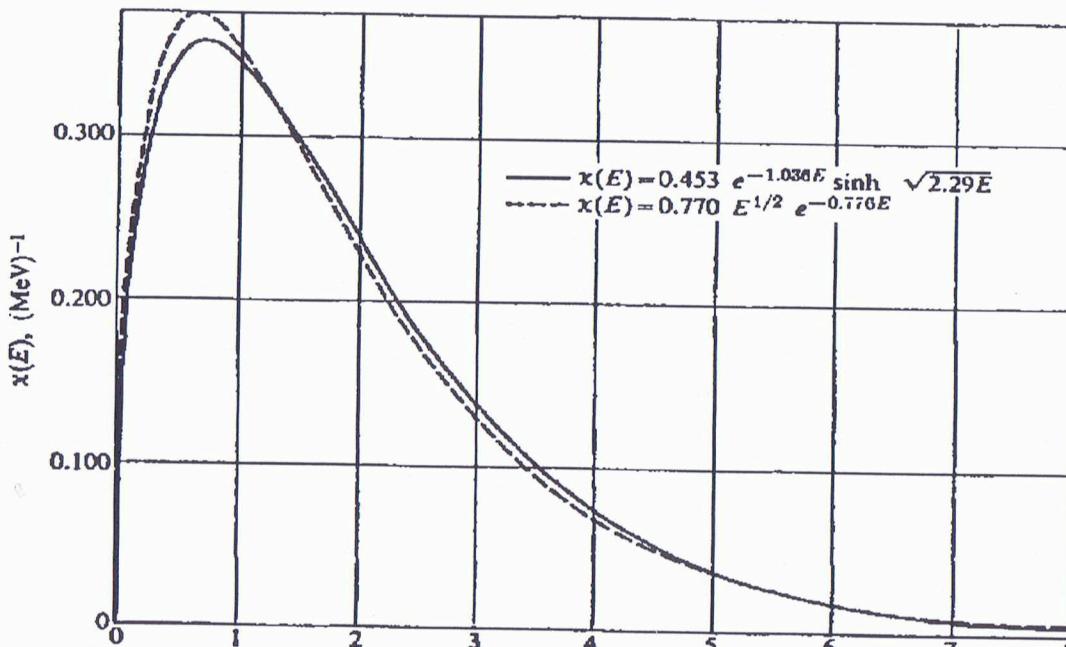
Neutron Energy

Fission is more effective at higher energies N
Smallest fission generation time at high energies ($T = 1 / \bar{\sigma}N \cdot v$)

Neutron Energy (MeV)

U_{235} Fission Neutron Energy Spectrum

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(Reference, Lamarsh, 1966)

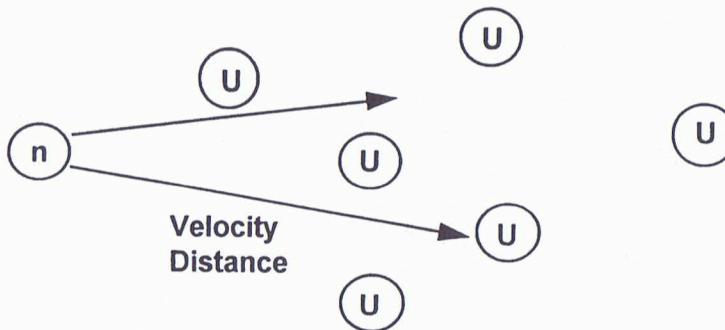
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"A Shake"



- Fission mean free path - how long before it clobbers an atom like URANIUM
 - Average velocity - how fast it is going
 - $\tau = \frac{\text{fission mean free path}}{\text{average velocity of neutron}}$
- These values are derived experimentally and are related to the fission cross section and velocity of the neutron.
- $\tau = 10^{-8}$ Seconds or 1 shake
(real fast like the shake of a lamb's tail)

We Care About Neutrons

- An efficient way to fission U²³⁵ or Pu²³⁹ is with neutrons.
- The fission of one atom of U²³⁵ or Pu²³⁹ releases approximately 200 MeV.
- To create an explosion by fission, a bunch of neutrons are required.
- The more neutrons--the more fission, i.e., We Care About Neutrons!
- Remember that each fission gives off integral numbers of neutrons--about 2-4, but over a bunch of fissions, we measure an average (i.e., 2.54 etc.) and this varies with input neutron energy.

\bar{v} = average number of neutrons

- The whole idea of sustaining the fission process is to get these fission neutrons to go fission more U²³⁵ or Pu²³⁹.
 - If all the neutrons escape without fissioning anything, then the reaction fizzles! (The population becomes extinct.)
 - If at least one of the 2 to 4 neutrons fission something every generation, then we have a steady state condition--a reactor.
 - If most of the neutrons fission another atom etc., etc., we have a run-away condition--a nuclear explosion.

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We Care About the Neutrons that Escape

-
- We call the escapees "lost neutrons," and the abbreviation is I (the letter after k).
 - So the number of neutrons available for population growth is the average number per fission (u), i.e., 2.54 minus the lost ones.
 - Someone called this k .
 - Therefore: $k = u - I$
 - for every neutron causing fission in one generation k will cause it in the next generation.

We Care About the Multiplication

- Now let's look at a bunch of fissions and bunch of neutrons.
- If we start with some number of neutrons (one or more), let that number equal n.

n = number of neutrons at beginning of a generation

- Remember, k = number of neutrons available for Round 2...
- And k times n equals number of neutrons at the next generation.
- Don't forget we've used up the original neutrons (n) in the first fission process..
- The gain of neutrons is thus:

$$n \cdot k - n$$

(number of neutrons we started with) • (average number in a fission of Round 2 (etc.)) minus the ones we used up in the previous round.

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Determine Growth Rate

- We still care about neutrons, but we really care about the rate (speed) that they are produced.
- The rate is the $\frac{\text{change in the number of neutrons}}{\text{change in time}}$
- Mathematically this is represented

$$\frac{Dn}{Dt} \longrightarrow \frac{dn}{dt}$$

- To get the rate change, we divide the actual gain in neutrons by time (t)

$$\frac{nk - n}{t}$$

- Therefore $\frac{dn}{dt} = \frac{nk - n}{t}$

Apply Basic Calculus

- $$\frac{dn}{dt} = \frac{nk - n}{\tau} = \frac{n(k - 1)}{\tau}$$

- Let α "alpha" $= \frac{k-1}{\tau}$ substitution gives

- $$\frac{dn}{dt} = n\alpha; \text{ Rearrange (cross multiply and divide)}$$

- $$\frac{dn}{n} = \alpha dt \quad \text{Integrate from zero neutrons } (N_o) \text{ to } N \text{ neutrons.}$$

- $$N = N_o e^{\alpha t}$$

If α is known, one can calculate the number of neutrons at any time (t).

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The Energy Released is Proportional to the Number of Fissions

$$\alpha \approx \frac{\mu - 1}{\tau} \approx \frac{3 - 1 - 1}{\tau} \approx \frac{1}{\tau} \text{ 1 gen / shake for 1 MeV neutron}$$

where: μ = ave# Neutrons
 ρ = Post Neutrons

$$N = N_0 e^{dt} \approx N_0 e^{\frac{t}{\gamma}} = e^g \text{ where } g = \text{Number of generations}$$

- The energy released is proportional to the number of fissions
- The number of fissions is proportional to the number of neutrons
- 1 fission $\approx 7 \times 10^{-21}$ tons of TNT
- At $g = 48$ we would have ≈ 9800 lbs.

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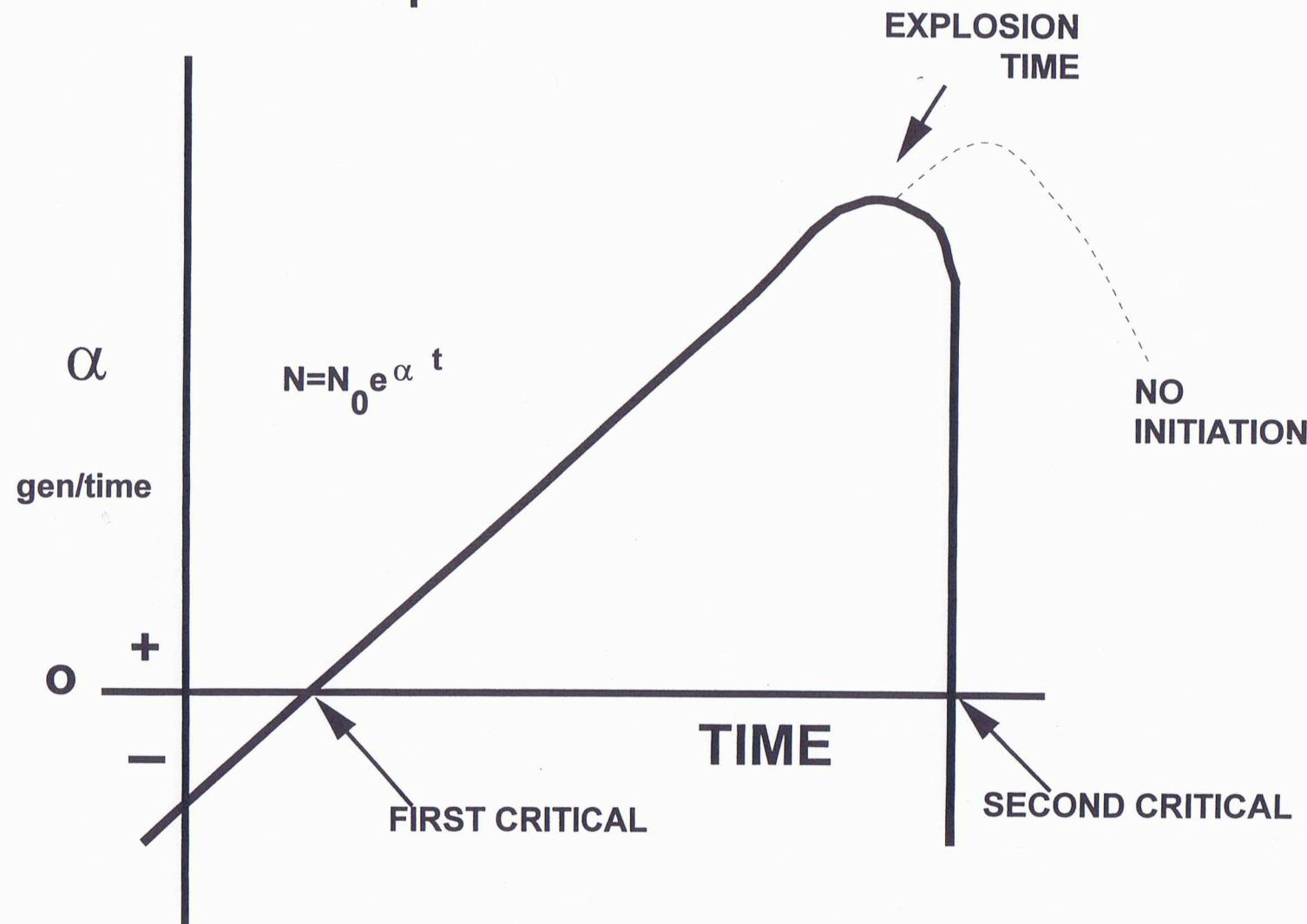
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α - Curve

Alpha



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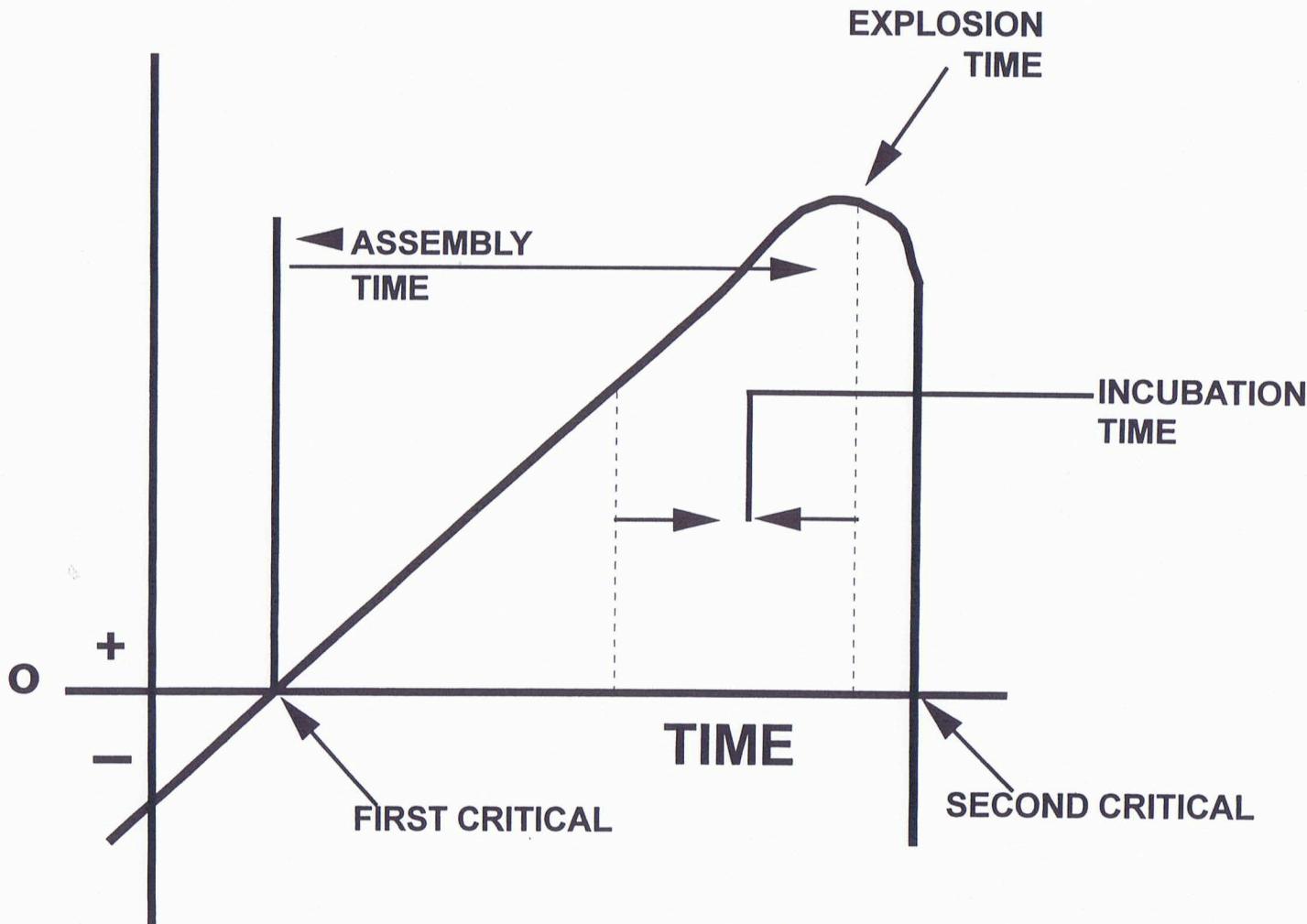
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α - Curve

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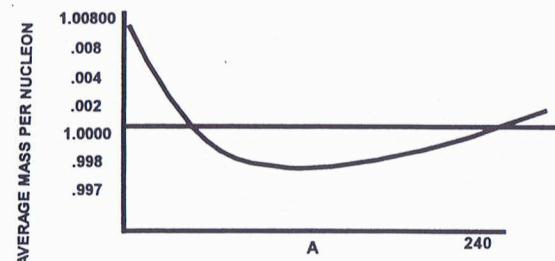
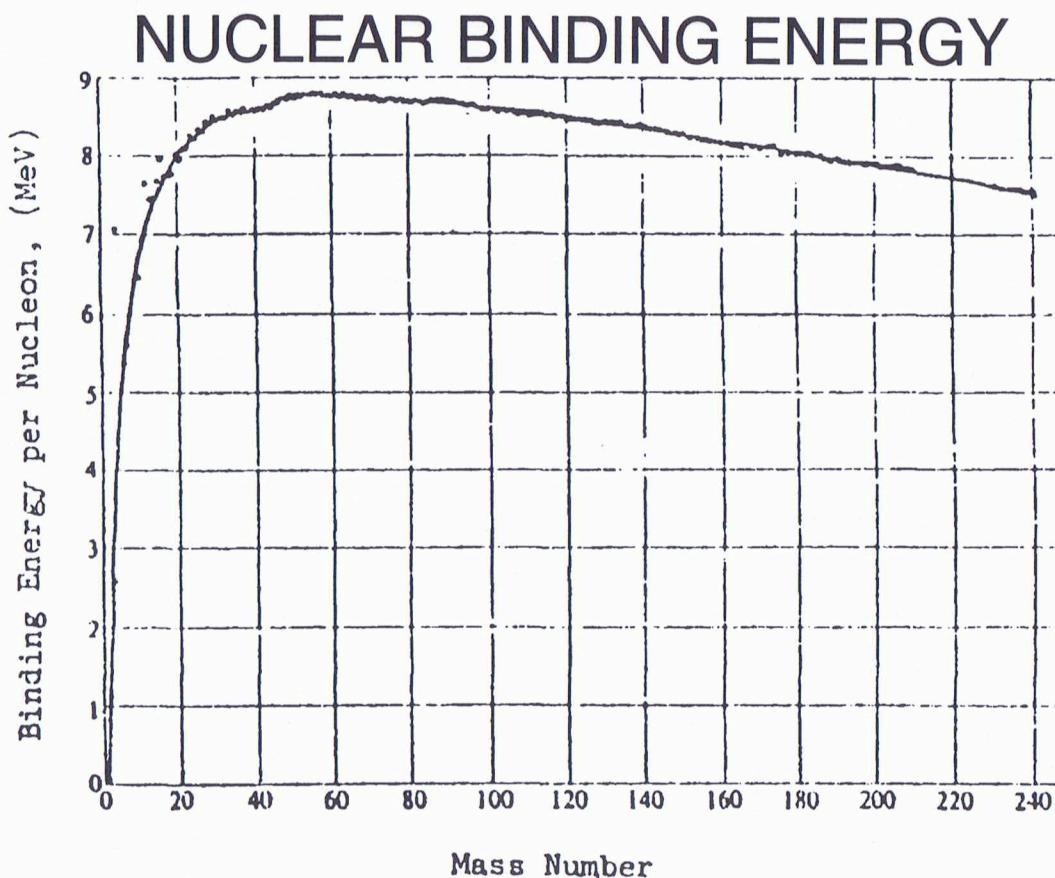
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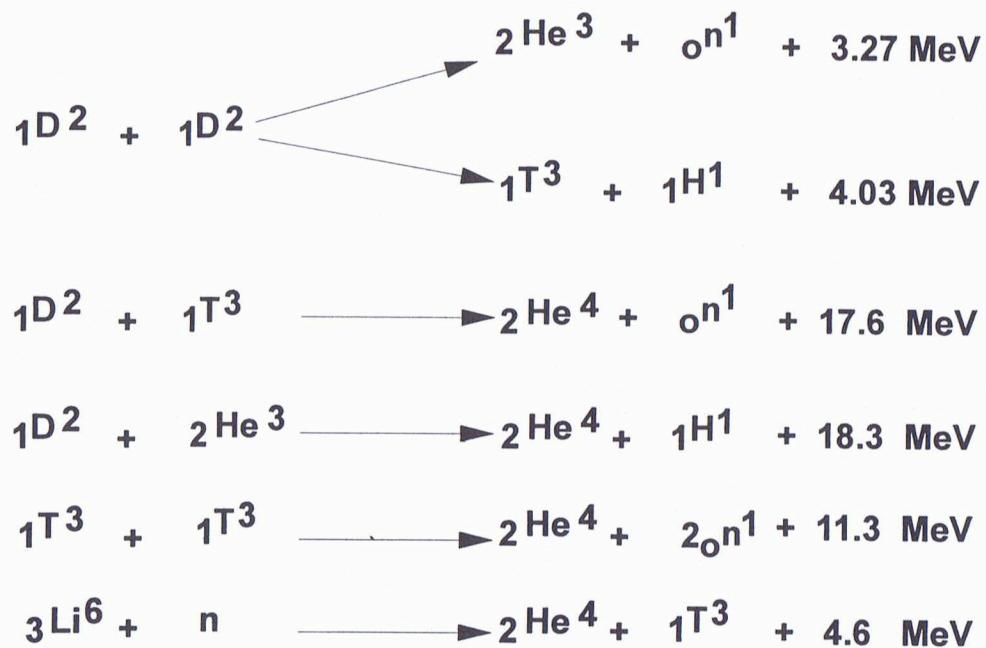
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Potential Fusion Reactions



Theoretical Fusion Energy in Equal Atom Mixture of Li⁶D

$$1 \text{ kg of Li}^6 \text{ has } \frac{6.025 \times 10^{26}}{6.0151} = 1.00165 \times 10^{26} \text{ Atoms}$$

$$1 \text{ kg of D has } \frac{6.025 \times 10^{26}}{2.0141} = 2.99141 \times 10^{26} \text{ Atoms}$$

Hence,

$$.25084 \text{ kg of D has } \left(\frac{2.01410}{6.01512 + 2.0141} \right) (2.99141 \times 10^{26}) \approx .7503841 \times 10^{26} \text{ Atoms}$$

$$.7491 \text{ kg of Li}^6 \text{ has } \left(\frac{6.01512}{6.01512 + 2.0141} \right) (1.00165 \times 10^{26}) \approx .750390 \times 10^{26} \text{ Atoms}$$

$$\text{Li}^6 + {}_1^0\text{n} \Rightarrow (.75039 \times 10^{26})(4.6)\text{MeV} \approx 13.2\text{kT}$$

$$\text{D} + \text{T} (.75039 \times 10)(17.6\text{MeV}) \approx 50.5\text{kT}$$

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TEMPERATURE EXPRESSED IN kT (ENERGY)

where K is Boltzmann Constant

$$1.38 \times 10^{-16} \text{ erg/}^{\circ}\text{K}$$

$$8.62 \times 10^{-8} \text{ keV/}^{\circ}\text{K}$$

$$T (\text{in keV}) = 8.62 \times 10^{-8} T (\text{in } ^{\circ}\text{Kelvin})$$

$$\text{Temperature of 1 keV} = 1.16 \times 10^7 \text{ degrees Kelvin}$$

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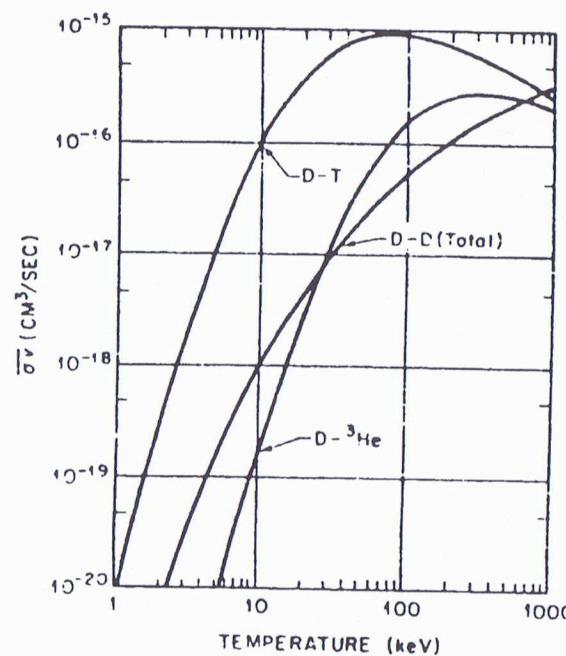
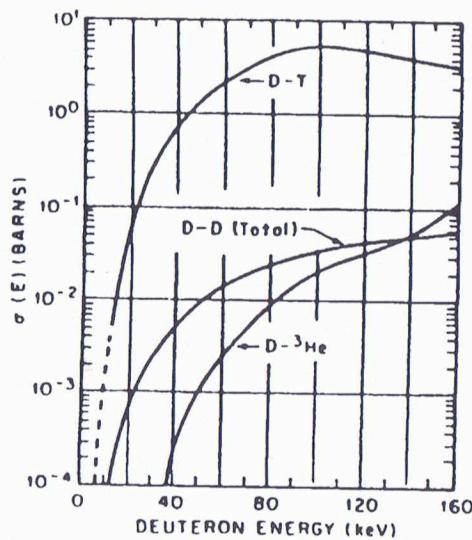
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Rational for Choice of Fusion Reaction

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FUEL

- ${}^6\text{LiD}$ (95% ${}^6\text{Li}$, 5% ${}^7\text{Li}$)

- Tritium



- Fusion



- Net Reaction



Net Energy = 22.3 MeV per Event

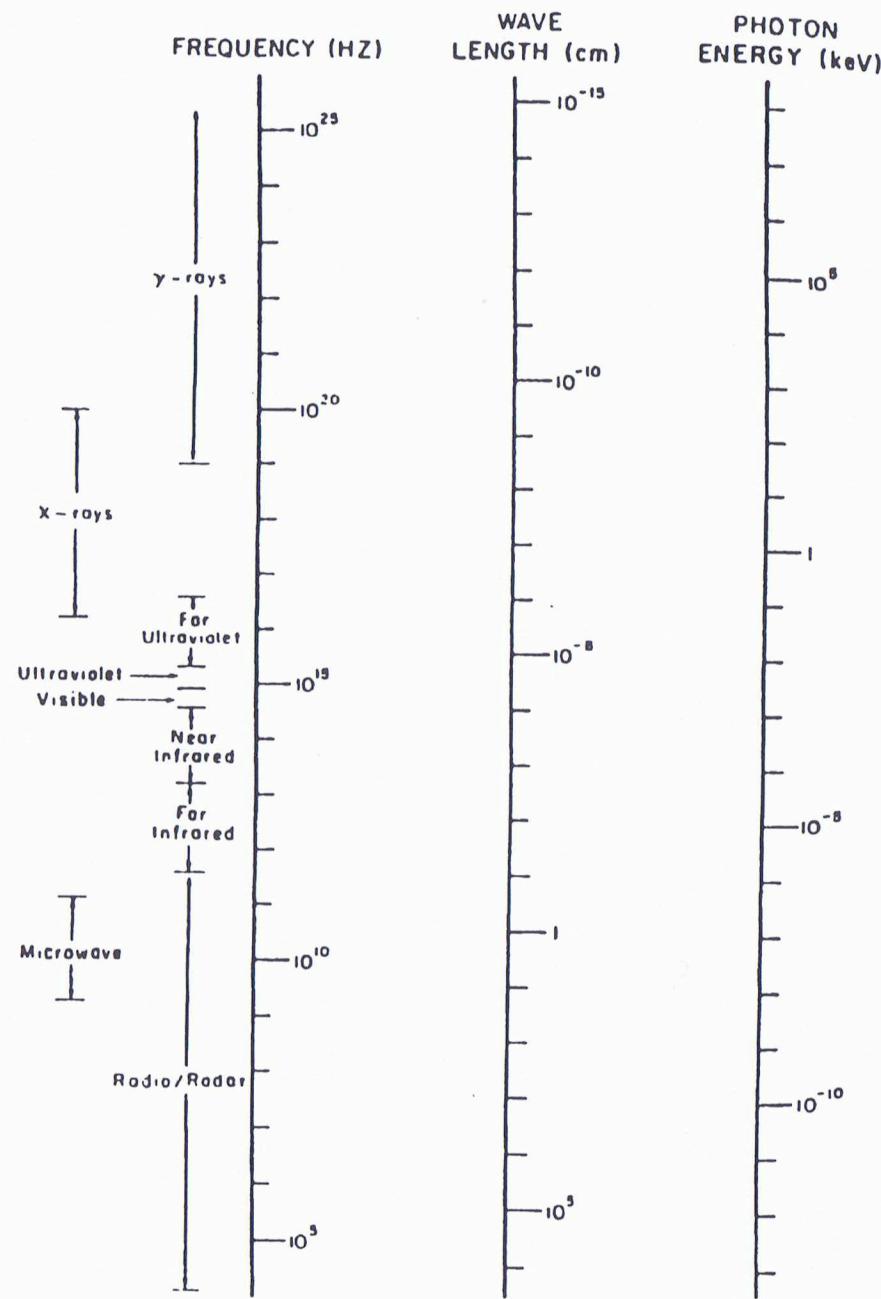
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Thermal Nuclear Plasma

AT FUSION TEMPERATURES, WE HAVE A PLASMA OF IONS (NUCLEI AND ELECTRONS).

$$\text{ENERGY} = aT_{(\text{ion})} + bT_{(\text{electron})} + cT^4_{(\text{radiation})}$$

IF PLASMA IS IN THERMODYNAMIC EQUILIBRIUM
THE THREE TEMPERATURES ARE EQUAL ➡ AT HIGH
TEMPERATURES, RADIATION WILL DOMINATE.

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REFERENCES

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